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**MULTI-AGENCY RADIATION SURVEY AND SITE INVESTIGATION MANUAL
(MARSSIM)
WORKGROUP MEETING NOTES - DRAFT**

MONDAY, SEPTEMBER 22, 2003

ATTENDEES:

U.S. Environmental Protection Agency, OERR/ERT	C. Petullo
U.S. Environmental Protection Agency, ORIA/HQ	K. Klawiter
U.S. Environmental Protection Agency, ORIA/HQ	L. Bender
U.S. Environmental Protection Agency, ORIA/NAREL	V. Lloyd
U.S. Environmental Protection Agency, Region 2	N. Azzam
U.S. Nuclear Regulatory Commission, RES	R. Meck
U.S. Nuclear Regulatory Commission, RES	G. Powers
U.S. Department of Energy (EM-33)	A. Williams
U.S. Department of Energy (EH-41)	E. Boulos
U.S. Department of Homeland Security (formerly DOE/EML)	C. Gogolak
U.S. Air Force	R. Bhat

MEMBERS OF THE PUBLIC:

Cabrera Services, Inc.	S. Hay (U.S. Air Force Contractor)
SC&A, Inc.	D. Schneider (NRC Contractor)

DISCUSSION:

Introduction, Agenda, and Objectives

C. Petullo welcomed the attendees, who introduced themselves to a new participant from EPA, L. Bender, and reviewed the agenda. The primary purpose of the meeting is to review Chapters 3, 4, and 5 of MARSAME to facilitate preparation of the internal agency review draft of the document by second quarter 2004.

Workgroup members were given an opportunity to provide agency updates. R. Bhat mentioned a document, prepared by S.Y. Chen of Argonne Laboratory, for NCRP. Other Workgroup members were familiar with the document, which is more related to regulatory and other technical issues beyond survey methods.

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Volumetric Contamination

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A. Williams presented a brief white paper on residual radioactivity in volume. He outlined three forms of volumetric contamination which the Workgroup discussed. The three forms of volumetric residual radioactivity were: 1) where radioactive materials (soil, rubble, etc.) have been piled up, loaded into trucks, or deposited into drums or boxes, 2) where radioactive materials have been used to construct property or where radioactive solutions or effluents have seeped into and contaminated porous materials, and 3) where nuclear particles have caused the contamination *in situ*; where neutrons or other particles have activated the substance to some depth.

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The first type of volumetric contamination that he identified occurs when radioactive materials, such as soil and rubble, have been piled, loaded into trucks, or deposited into drums or boxes. Such a scenario poses a potential problem in accessing the interiors of the waste volume from the surface. Detectors might be possible to use, but the thicker the material or the larger the pile, the less the instruments can detect contamination within the pile. In addition, naturally occurring radioactivity may serve as a confounder. He advised trying to avoid such a scenario in the first place by surveying the material before piling it, when the material is spread out and the surfaces are accessible, or sample it as it is being loaded.

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If the material has already been piled or loaded into containers, implementing a specific protocol for volumetric contamination and hot spot detection can still be avoided by removing the material and spreading it out to measure its surface. For piles that cannot be spread, an iterative process can be applied where the surface of the pile is measured, a scoop of material removed for loading into a container, and the pile measured again.

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R. Meck commented that such guidance should include visual inspection for sources in the material. To address nuclides that are difficult to detect, the guidance should discuss implementing a risk-informed process based on applicable regulatory requirements. Risk considerations present challenges to the development of the guidance because in addition to the technical considerations, the survey then must be considered in the context of regulation. A. Williams described an example of material in drums, where process knowledge can be used to determine what radionuclides are expected to be present and if the contamination is expected to be homogenous. Depending on what level of data are needed to release the material to a waste facility, measuring the drums with a drum counter may be sufficient. If the waste is not expected to be homogenous, or if strong sources are not expected, then the material would have to be spread out to measure the surface directly. It is more efficient to characterize prior to containerizing due to safety, cost, and dose considerations. G. Powers noted that such a process was actually characterization in preparation for selecting measurement types and methods. It is part of the historical assessment and decision process.

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The Workgroup then discussed the second form of volumetric contamination considered by A. Williams, where radioactive materials have seeped into and contaminated porous materials, such

72 as building materials. It is particularly relevant with regard to tritium contamination. While the
73 Workgroup decided that this scenario is not substantively different than the first one, it does
74 present a situation where there might be unusual isotopic characteristics and more difficulty in
75 determining the maximum amounts of radionuclides that would be encountered. Contamination
76 in building materials can be shielded by other materials, making volumetric contamination more
77 difficult to determine. Areas of contamination should be identified and analyzed separately from
78 materials not affected. Guidance is needed on the amount of coring and other measurements and
79 sampling that is sufficient to find contamination. However, the Workgroup determined that once
80 the materials are evaluated, sorted, and piled, they become covered under the first case.

81 R. Meck noted that such scenarios represent a departure from an “ideal” scenario and require an
82 increasing reliance on historical assessment. Accompanying guidance should indicate how much
83 reliance on historical assessment is too much and when the material must be spread out and
84 measured to be cleared. Regulatory issues, such as waste acceptance criteria, will often drive the
85 level of knowledge that one must have of the potential contamination in the piles to clear it. R.
86 Bhat noted that while gamma contamination might be dealt with in this way, alpha or beta
87 contamination is more difficult to detect cost effectively. R. Meck outlined what might be
88 considered the elements of an “ideal” or “easy” case, such as good historical assessment, the
89 presence of a gamma emitter, and the ability to conduct a visual inspection. He suggested that
90 the guidance begin with such an example, then lead to more challenging scenarios where one or
91 more elements are not possible.

92 Measurement Difficulty Hierarchy: Easiest

- 93 C Materials: spread out
- 94 C Visual inspection: yes
- 95 C Gamma measurements: yes
- 96 C Historical assessment: high quality (optimal would be if the inventory includes
97 maximum actual contamination)
- 98 C Homogeneity: yes

99 If one of the above elements is missing, the other aspects must be stronger in order to
100 compensate, particularly the historical assessment. These points will be expanded on in the
101 section on volumetric contamination, as well as highlighted where they already exist in the
102 document, since they can also apply to instrument and measurement selection for non-volumetric
103 contamination.

104 A. Williams’ third case involves nuclear particles which have caused contamination *in situ* by
105 activating the substance to some depth. This case would require substantial evaluation, through
106 coring or other analyses, to find non-gamma emitters. The distribution of radioactivity would not
107 be completely evident and presents the potential for non-homogenous contamination. Cases of
108 induced radioactivity at depth are especially relevant with regard to particle accelerators and
109 nuclear reactors.

110 The Workgroup considered what other analyses would be appropriate for such cases. Lessons
111 learned from similar facilities, surrogate analyses, and modeling of the fluence rates of neutrons
112 could be used to help determine where contamination might be expected and to take judgmental
113 samples. Ideally, samples should be collected before items are turned to rubble, or before they
114 are piled. If this is not possible, sampling cannot be avoided. Hard data are required to satisfy
115 most regulations, and compliance is based on tangible actions and evidence, as described in the
116 paper on the Daubert factors. C. Petullo will ensure that all Workgroup members have a copy of
117 the paper and will ask the author to speak at a Workgroup meeting. V. Lloyd also suggested
118 Steve Frye at Paragon Laboratories in Ft. Collins, CO, as another good source of information on
119 this topic.

120 Issues to keep in mind and focus on when addressing volumetric contamination, including the
121 selection of measurement techniques and historical assessment, will be presented in S. Hay's
122 chapters of MARSAME. R. Meck suggested that the beginning of each section will focus on an
123 ideal case and then break down the areas of potential difficulty as the reader moves through the
124 section. The discussion of volumetric contamination will relate to the level of certainty in taking
125 measurements, assuming that surfaces can be measured while deeper contamination cannot be
126 measured from the surface.

127 FAQ and MDC Development Update

128 C. Gogolak, is now officially a member of the Workgroup as the representative of the
129 Department of Homeland Security.

130 He updated the Workgroup on his progress in developing the FAQs and the MDC discussion. R.
131 Coleman provided him with factors that are important with regard to uncertainty, and the rest of
132 the Workgroup should also provide input. At the request of R. Meck, Harry Chmelynski of
133 SC&A prepared two papers during MARSSIM development that are relevant to C. Gogolak's
134 work. The first discussed analyzing multiple radionuclides by using surrogates when the
135 radionuclides are related and applying the Unity Rule when they are not. The paper, which
136 relates to Appendix I, mainly considered the situation when there is not a distinct relationship
137 and some uncertainty exists. It examined the level of certainty needed when applying the ratios.
138 For use in MARSAME, the discussion should also address the derivation of the ratios
139 themselves. Uncertainties exist due to the variability of ratios, but no uncertainty is assumed in
140 MARSSIM when dealing with radionuclides that are exactly related.

141 The other paper covered instrument MDC for the final status survey design. When given a fixed
142 grid size and measurement criteria, a certain MDC is implied, and the real DCGL can be
143 determined. Whereas the first paper looked at ratios of radionuclides to propagate the error
144 based on real data, the second paper considered the ratio of the MDC to the DCGL. The MDC
145 tells if radiation is present in the sample. Action is taken if it is present, and no action is taken if
146 it is not. The amount of radiation present does not matter. In MARSSIM, detection itself is not
147 the criterion, but whether the amount detected is above or below a certain level within some

148 preset degree of uncertainty. Therefore, the Workgroup was considering a test where the MDC
149 was less than the DCGL, which led to the maximum quantifiable concentration (MQC), which is
150 the concentration at which the relative uncertainty is $\pm 10\%$. The MDC divided by the DCGL
151 should be less than one, and the smaller it is the better. The paper related the impact of different
152 levels of the MDC on the relative shift and would be helpful if the conclusion is clarified.
153 Therefore, C. Gogolak will assist R. Meck in directing Harry Chmelynski on how to proceed
154 with finishing the papers in September.

155 C. Gogolak also reported that the overview of MARSSIM which he presented at a meeting of the
156 American Chemical Society generated much interest. At the request of Cheryl Trotter, he will
157 give a similar presentation at the NRC Light Water Reactor meeting in October. He will
158 continue working on the MDC and other FAQs and the MDC chapter. A conference call with
159 the Workgroup to discuss the draft FAQs will be held sometime between the meeting with the
160 SAB in October and the December Workgroup meeting; a date will be set on October 21. The
161 SAB meeting will take place on October 21, possibly followed by a meeting of the Workgroup
162 on October 22.

163 SAB-RAC Presentation Development

164 Using the presentation given previously to the SAB-RAC, the Workgroup began developing the
165 presentation for the October 21 meeting, which will cover the key issues that the SAB raised at
166 the previous meeting. The meeting will extend for a full day on October 21 and conclude the
167 next morning. At the suggestion of V. Lloyd, the presentation will open with a review of the
168 Workgroup's questions to the SAB and the concerns described by the SAB at the previous
169 meeting. It will then cover the progress to date in resolving the questions and what remains to be
170 done.

171 The Workgroup began by refining the purpose of MARSAME, stating that it is intended to
172 provide guidance on measurements for release (rather than administrative guidance) to optimize
173 a release survey protocol (rather than to develop an efficient protocol), considering certain issues.
174 With regard to material being released, the Workgroup added the nature of the potential activity
175 and other text to convey volumetric contamination. To address the SAB's question about the
176 relationship between MARSSIM and MARSAME, the Workgroup used text from K. Klawiter's
177 presentation to the Health Physics Society. A new slide will show progress and clarifications
178 since the previous meeting. Progress will include a mention of FAQ development relevant to
179 MARSSIM. C. Gogolak will provide some bullet points on the FAQs for the presentation.

180 The flow charts will be included in the presentation, labeled as working drafts, to address the
181 SAB's request for a roadmap. The Workgroup discussed revisions to the flow charts. The boxes
182 considering whether the MDC is less than or greater than the DCGL, and whether an item passes
183 or fails, apply to both accessible and inaccessible areas. N. Azzam was tasked with revising the
184 flow charts and ultimately finalizing them using the regular format for decisions and results in
185 flow charts.

186 The Workgroup reviewed the minutes from the SAB meeting to identify topics, including: the
187 use of modeling and how it interfaces with the document; monitoring of inaccessible areas;
188 restricted v. unrestricted release of materials; volumetric v. surface contamination; sentinel areas;
189 and orphan sources and TENORM (serving as confounding factors).

190 The Workgroup discussed the SAB's comment regarding the lack of a free-release scenario for
191 clearance in MARSAME. At the October meeting, the Workgroup will ask the SAB for
192 clarification on whether it was requesting an example of a clearance scenario, or if the comment
193 referred to MARSAME's interface with modeling and possible problems that may be
194 encountered during the release survey. V. Lloyd noted that many SAB comments related more to
195 reuse than other types of release.

196 The Workgroup will note in the presentation that it does not agree with the SAB's comment that
197 a discussion on when and how to use smears for release should be included. Smears are defined
198 in MARSSIM as a diagnostic tool to determine if further investigation is necessary, because
199 smears do not produce quantitative results.

200 More detail than originally planned will be added to the document on accurate monitoring and
201 data quality, while at the same time avoiding prescriptive guidance.

202 The Workgroup will develop functional scenarios that better define the problem and issues and
203 are more specific with regard to monitoring and measurement methods.

204 To address another of the SAB's concerns, S. Hay has already added some text on sentinel areas.

205 The Workgroup considered the SAB's comment on moving from measured data to a dose
206 estimation. If the probability of missing some contamination is increased, the risk of doing so
207 should be quantified and factored into the decision criteria. However, such an approach would
208 be up to the regulator and therefore out of the MARSAME scope.

209 The SAB expressed concern about people picking up alpha contamination on their clothing from
210 radon decay products and causing a false positive. Since those who perform scan-out surveys are
211 aware of this issue already, the Workgroup does not feel that it bears more than a brief mention.

212 The SAB also asked about accelerators. Because there is not a single regulatory structure for
213 accelerators, MARSAME will cover this topic through its overall approach.

214 The Workgroup noted that a reference area may be the object of concern itself. A survey of the
215 object should be performed prior to its use in a radiation area. This information would then be
216 used as the reference area for the final status survey. This represents a significant difference
217 from MARSSIM, since the ability to survey an object before it becomes contaminated was not
218 possible under MARSSIM.

219 R. Meck noted that the inclusion of more examples in MARSAME would be desirable, and
220 several possibilities should be mentioned in the presentation. The SAB should be asked for
221 feedback on which three would be the most useful. The examples include the following:

- 222 C university of research and development laboratory (with isotopes typically used at a
223 university)
- 224 C nuclear accelerator
- 225 C hospital facility (trash)
- 226 C pile of demolition debris
- 227 C personal property from nuclear power plant (operations and closure)
- 228 C reuse of equipment
- 229 C mine/mill property
- 230 C military - depleted uranium (DU)

231 Scenarios could be combined. For example, the reuse of equipment could be addressed in a
232 nuclear power plant or other setting. The impact of radon on surveys in general should be
233 covered. The Workgroup noted that releases related to operations are not different than those
234 related to closure. A. Williams will identify the nuclides associated with each of the scenarios to
235 facilitate the selection of scenarios to combine.

236 R. Meck suggested that each example should teach a specific lesson as part of the scenario, such
237 as the sum of fractions or use of conveyor belts. Therefore, the Workgroup should ask the SAB
238 for advice on what principles, concepts, processes, and techniques for each example most need to
239 be conveyed.

240 Using the Workgroup's input, C. Petullo developed a draft presentation during this week's
241 meeting to provide to the Workgroup for comment on Friday, September 26.

242 TUESDAY, SEPTEMBER 23, 2003

243 ATTENDEES:

244	U.S. Environmental Protection Agency, OERR/ERT	C. Petullo
245	U.S. Environmental Protection Agency, ORIA/HQ	K. Klawiter
246	U.S. Environmental Protection Agency, ORIA/HQ	L. Bender
247	U.S. Environmental Protection Agency, ORIA/NAREL	V. Lloyd
248	U.S. Environmental Protection Agency, Region 2	N. Azzam
249	U.S. Nuclear Regulatory Commission, RES	R. Meck
250	U.S. Nuclear Regulatory Commission, RES	G. Powers
251	U.S. Nuclear Regulatory Commission, NMSS	A. Huffert
252	U.S. Nuclear Regulatory Commission, NMSS	J. DeCicco
253	U.S. Department of Energy (EM-33)	A. Williams
254	U.S. Department of Energy (EH-41)	E. Boulos
255	U.S. Department of Homeland Security (formerly DOE/EML)	C. Gogolak
256	U.S. Air Force	R. Bhat

257 MEMBERS OF THE PUBLIC:

258	Cabrera Services, Inc.	S. Hay (U.S. Air Force
259		Contractor)
260	Oak Ridge National Laboratory	R. Coleman (DOE
261		Contractor)
262	SC&A, Inc.	D. Schneider (NRC
263		Contractor)
264	SC&A, Inc.	R. Abedin (NRC Contractor)

265 DISCUSSION:

266 Review of Chapter 3 of MARSAME

267 The Workgroup worked through Chapter 3, discussing the text as well as the questions posed by
268 S. Hay during the development of the chapter.

269 The Workgroup discussed the relationship between regulatory, administrative, and institutional
270 limits in Section 3.1 on release limits. Administrative limits are often, but not always, set
271 according to regulatory limits. Action guidelines and the amount of work required for
272 compliance with them will vary based on their stringency and the complexity of the task.
273 MARSAME should not focus on the idea of limits but rather relate release limits to the intended
274 use of the materials and emphasize the fact that they serve as action guides. R. Meck noted that
275 the release of materials and equipment into general commerce should not be referred to in terms

276 of “protect(ing) human health and the environment,” because the radiation levels at issue in
277 clearance are already much lower than the 100 mrem/year limit already in place to protect health.
278 V. Lloyd , C. Gogolak, and R. Meck worked together to prepare new draft text for presentation to
279 the Workgroup on Wednesday.

280 S. Hay noted that he used the term “release limits” rather than “release criteria,” as was used in
281 MARSSIM, since MARSSIM defined a release criterion as a regulatory limit. The term “release
282 limit” is broader and can include administrative and institutional limits as well as regulatory
283 limits. This terminology must be consistent throughout MARSAME and the term “release limit”
284 defined in the glossary.

285 An example about the concentration of radionuclides during incineration will be added to Section
286 3.1, lines 41-44. A. Williams will provide information and data on incineration to R. Meck.

287 C. Gogolak had concerns about the discussion of DCGLs, which used the definition of DCGL
288 from MARSSIM. In the case of MARSAME, the DCGLs are also based on the transport of
289 material and how individuals are exposed to it. Once the term “DCGL” is defined for the
290 purpose of MARSAME, the definitions of “DCGL_C,” “area factor,” and “volume factor” will be
291 easier to develop, as well as their relationship to the DCGL_{EMC}.

292 The beginning of Chapter 3 should note that changes in definitions will occur as scenarios
293 become more complex. At the most basic level, a survey unit is that which will be the object of a
294 decision. In order to make that decision, certain data must be found, and MARSAME will
295 describe how to measure and obtain the data. However, once the actual survey unit is defined for
296 practice, the situation will become more complicated. S. Hay will describe the derivation of the
297 DCGL from the risk- and dose-based limit, as well as other subsections, including defining the
298 survey unit and the interface with modeling, in more detail. Modeling will be described in terms
299 of its use in conjunction with the release limits to derive a DCGL. The text will emphasize that
300 MARSAME is a measurement document, although it should mention issues involving the
301 appropriateness of the DCGL, given the ultimate use of the released material. The definition of
302 area factor from the MARSSIM glossary should be included. It was suggested that the
303 discussion of F_T equal to F_M and F_s approaches 0 for *in toto* and scan only surveys be moved to
304 an appendix, along with a discussion of the specifics of survey design.

305 The Workgroup discussed the material to be covered in Section 3.2.3, on gaseous materials. The
306 section should cover containerized gases, rather than discharges of gases to the atmosphere, just
307 as the previous section on liquids covers liquids in a container, not those released into a stream,
308 for example. Accelerators and research reactors are most likely to deal with containerized gases.
309 The guidance should address gases in terms of how to measure gases in a container for release of
310 the container. While measurement with a gamma spectrometer may be possible, in some cases
311 the gas in the container will have to be sampled directly.

312 Uncontained gases will not be covered in terms of releasing the gases themselves. Instead,
313 MARSAME should mention that non-equilibrium radon issues, such as radon’s role as a
314 confounder and its impact on a survey unit, must be taken into account, recognizing that this is a
315 modeling issue and not a measurement topic. Radiation emissions can be absorbed into other
316 materials through an airborne pathway and these effects can be modeled. However, the
317 measurement of contamination and the release decision itself will be based on the solid that
318 absorbed the gaseous radiation, making it a contaminated solid. The release of uncontained
319 gases is handled in the context of effluent regulations, such as NESHAPs.

320 R. Meck noted that Section 3.3 on survey units should be connected back to understanding the
321 derivation of the DCGL. He suggested referring readers to NUREG-1640 for examples of survey
322 units. Since not all Workgroup members are experienced in the application of NUREG-1640, R.
323 Meck will present that topic to the Workgroup at the December meeting. R. Bhat mentioned that
324 including guidance on average, recommended maximum, and recommended minimum survey
325 unit sizes would be helpful, but the Workgroup noted that such suggestions and examples in
326 MARSSIM were taken as requirements and applied directly by MARSSIM users. C. Gogolak
327 explained that while NUREG-1640 could be given as an example of how to determine a dose-
328 based scenario, not all MARSAME users will be working within a dose-related regulatory
329 framework. Therefore, users must understand how to apply the size of the survey unit to cases
330 where NUREG-1640 is not applicable. He suggested providing a detailed example of how the
331 users could find the appropriate value themselves. R. Bhat suggested that detailed pros and cons
332 outlining two different approaches to the same problem in a practical example would be useful in
333 showing MARSAME users how to adapt the guidance to their own situations. The Workgroup
334 agreed that the approach for defining a survey unit would be given in Section 3.3 and described
335 in more detail later in the document. Given the propensity of users to use numerical examples as
336 de facto guidance, the Workgroup agreed to delete Table 3.1 on typical material survey unit
337 sizes.

338 The Workgroup then discussed the difference between sentinel and surrogate measurements in
339 Section 3.4. Surrogate measurements refer to the correlation between what can be measured in
340 one case with what is more difficult to measure in another. For example, the Workgroup
341 considered a large pump as a survey unit. Measurements made on one side of the intake or
342 turbine can be used as surrogates for measurements on the other side. In addition, certain
343 radionuclides can also be used as surrogates for other radionuclides. Sentinel measurements refer
344 to key locations in different scenarios where contamination would be concentrated if it were
345 present. If measurements are made in these places and contamination is not found, the entire
346 object is probably not contaminated. S. Hay will elaborate on these distinctions in the document.
347 This issue will be revisited with S. Doremus, who had originally raised the question, at the next
348 meeting.

349 The Workgroup moved on to Section 3.5. Because MARSAME can be used for contaminations
350 in solids, liquids, or gases, the references to “solid” materials in the section will be removed.
351 When defining Classes 1, 2, and 3, S. Hay will ensure that they are consistent with the updated

352 definitions from MARSSIM which are on the MARSSIM website. For Section 3.5.4 on special
353 considerations for small activity quantities and short half-lives, R. Meck will provide a paragraph
354 on a paper he prepared, which discusses the derivation of an arbitrary probability that all of a
355 particular radionuclide with a small activity quantity or short half life is gone. This approach can
356 be used as a short cut to determining the presence of contamination, where time is measured
357 instead of concentration.

358 Section 3.6 describes the selection of background reference materials and equipment. The topic
359 is currently introduced in Chapter 2. R. Meck suggested moving the overview of background
360 radiation from Chapter 2 to an appendix. Section 3.6 should cover practical guidance and
361 specifics about issues that may arise, such as details of activity levels that might be expected and
362 the selection of reference areas. The table currently in Chapter 2 will be moved to Section 3.6.

363 The Workgroup discussed the inclusion of a table in Section 3.7 on the selection of instruments
364 and survey techniques. The Workgroup decided to include the parts of MARSSIM Table 4.1 that
365 cover the measurement of the surfaces of structures and combine it with the information on
366 measurement technologies for volumetric contamination from the third page of Table B-3a from
367 NRC draft NUREG-1761, provided by G. Powers. S. Hay will prepare the table, including
368 standard units in addition to Bq/m². G. Powers suggested developing an appendix on
369 instrumentation that is complete and easily updated.

370 For Section 3.7.3, the Workgroup discussed the types of homeland security issues which would
371 be covered by MARSAME. MARSAME will not be applicable to measuring radiation on
372 people. R. Meck noted that in terms of screening levels for dispersed sources (not concentrated
373 sources that are shielded), the MDCs for clearance should be satisfactory for homeland security.

374 [Discussion of Chapter 3 continued after the Scrap Metal presentation (pg. x)]

375 EPA Detection of Radioactive Material in Imported Scrap Metal Pilot Project: Presentation by
376 Sally Hamlin and David Kappelman, EPA

377 Sally Hamlin, EPA, ORIA/RPD, and David Kappelman, EPA, ORIA/NAREL, presented to the
378 Workgroup an overview of an EPA project with the Bureau of Customs and Border Protection
379 that relates to detecting radiation contamination in large volumes of material. The project was
380 originally developed because of the effects of orphan sources of radiation on commerce.

381 At the request of Congress, the Bureau of Customs and Border Protection initiated a study with
382 EPA to monitor imported scrap metal entering U.S. ports for radioactive contamination.
383 Radioactive sources in scrap metal can be melted in with the metal, resulting in the
384 contamination of consumer metal supplies, significant financial impact for the melt facilities, and
385 health risks for their workers and the general public. Orphan radioactive sources that have fallen
386 out of regulatory control have been detected in scrap shipments in western Europe, so the United
387 States is focusing initially on imported scrap metal shipments.

388 The project is intended to determine the amount of contaminated metal in the United States that
389 comes from contaminated imports to determine if development of a standard is necessary. The
390 project evaluates the feasibility of mounting radiation detectors directly onto the equipment used
391 at the off-loading ports, by developing a pilot program of detection and response protocols with
392 industry participation that may be expanded to other ports and maintained by industry. The pilot
393 project is currently in place at a port in Darrow, Louisiana, and at the Port of North Charleston,
394 South Carolina. All scrap metal imported is monitored at each facility, and the system is in use
395 24 hours a day, seven days per week.

396 The project requires evaluating scrap delivered in large freighters, with approximately 65,000
397 tons of scrap in the hull of each ship. The density and type of material vary, and portal monitors
398 and other common measurement methods are not effective. In order to accurately detect
399 radiation, the total volume of scrap must be divided into smaller volumes during sampling to
400 increase the probability of detection. The position of the source and the variation in the density
401 of the material will affect the level of activity detected. The detection method must have
402 minimal interference with commerce. Equipment must be rugged. Finally, the equipment and
403 sampling protocols must be validated and repeatable.

404 To achieve these requirements, EPA selected the Rad/Com Cricket system, manufactured by
405 Rad/Com Systems in Ontario, Canada. The system is a large area detector which fits in the base
406 of the grapple used for off-loading scrap metal from the freighter's hulls. The grapple varies in
407 size between eight and 15 yards. The detector consists of two plastic scintillation detectors with
408 two photomultiplier tubes surrounded by high strength protective shielding. The detectors,
409 approximately 2' x 4' x 6" in size, communicate with the alarm system without wires. The
410 detector evaluates each load of scrap as it is carried by the grapple from the freighter hull. If
411 radiation is detected, an alarm sounds for the crane operator as well as in another selected
412 location. The crane operator then separates that load from the rest and continues working. The
413 load is evaluated separately to determine the nature of the contamination. If two loads in a row
414 set off the alarm, the entire freighter load is considered suspect for contamination. Daily quality
415 control checks are performed by stevedores to ensure that the system remains functional.

416 Data are logged every five seconds in the form of counts at each time. EPA is using the data to
417 verify that daily QC checks are performed. If positive readings were found, the data could be
418 evaluated but still would not provide much information. In the future, standard operating
419 procedures will require data logging at the beginning of every shift, but not data logging of the
420 entire operation. There is no separate QC log for the pilot program.

421
422 Both neutrons and gamma radiation can be detected and set off the system's alarm. However, the
423 system does not differentiate between the two or provide any information on what radionuclide is
424 present. Each grapple load is considered a survey unit, and a decision is made for each grapple
425 load based on whether the alarm goes on or not as a result of the direct measurements made by
426 the system.

427 The industry is currently implementing the project, with oversight from EPA. No positive
428 detections for radioactive scrap have been found by the pilot project to date, but several positive
429 detections have occurred in Europe using the same system. The pilot project is being evaluated
430 for use at other ports of entry.

431 Review of Chapter 3 of MARSAME (Continued)

432 After the presentation and discussion, the Workgroup returned to its review of Chapter 3. The
433 current draft of Section 3.7 covers only MDCs; S. Hay asked the Workgroup to identify other
434 aspects of instrument selection that are different from what was discussed in MARSSIM and
435 should be included in MARSAME. V. Lloyd suggested that issues such as the physical nature of
436 the materials, the suspected nuclides of contamination, and whether or not a scan was possible
437 should be discussed first in the section before covering MDCs. Many of these topics are already
438 in Chapter 3 in the sections on solids, liquids, and gases. R. Coleman noted that Chapter 4, on
439 survey planning and design, also began by considering these topics, while S. Hay thought that
440 Chapter 4 should be more concerned with the number of samples and other such issues. Because
441 of the number of issues involved, the Workgroup decided to continue discussion of Section 3.7
442 on Wednesday.

443 Section 3.8.2, on survey difficulties, includes a discussion of homogeneity. The Workgroup
444 noted that striving for homogeneity in the normal course of identifying a survey unit is
445 acceptable, but using it to dilute the contamination present would not be. The guidance should
446 state that the technique can be used to make surveying easier, but heterogenous items should not
447 be intentionally added to make something that is contaminated pass. Impacted and nonimpacted
448 materials should not be mixed together; items that have already been identified as Class 1, Class
449 2, or Class 3 should not be mixed with other classes.

450 The Workgroup discussed topics for Section 3.9, on special considerations for quality control.
451 This section should cover the visual inspection process and the identification of orphan sources,
452 and possibly the supplementation of a scan of less than 100 percent with measurements.

453 MARSAME does not expand on the reference coordinate system and health and safety, which
454 were covered in MARSSIM. The Workgroup agreed to decide whether or not to cover these
455 topics on Wednesday.

456 WEDNESDAY, SEPTEMBER 24, 2003

457 ATTENDEES:

458	U.S. Environmental Protection Agency, OERR/ERT	C. Petullo
459	U.S. Environmental Protection Agency, ORIA/HQ	K. Klawiter
460	U.S. Environmental Protection Agency, ORIA/NAREL	V. Lloyd
461	U.S. Environmental Protection Agency, Region 2	N. Azzam
462	U.S. Nuclear Regulatory Commission, NMSS	J. DeCicco
463	U.S. Nuclear Regulatory Commission, RES	R. Meck
464	U.S. Department of Energy (EM-33)	A. Williams
465	U.S. Department of Energy (EH-41)	E. Boulos
466	U.S. Department of Homeland Security (formerly DOE/EML)	C. Gogolak
467	U.S. Air Force	R. Bhat

468 MEMBERS OF THE PUBLIC:

469	Oak Ridge National Laboratory	R. Coleman (DOE
470		Contractor)
471	Cabrera Services, Inc.	S. Hay (U.S. Air Force
472		Contractor)
473	SC&A, Inc.	R. Abedin (NRC Contractor)

474 DISCUSSION:

475 Review of Chapter 3 of MARSAME (continued from Tuesday, September 23)

476 Supplemental Information for Insertion in Chapter 3

477 The Workgroup continued its review of Chapter 3, discussing the text as well as the questions
478 posed by S. Hay during the development of the chapter. V. Lloyd presented a new paragraph on
479 “release limits” to insert in Section 3.1, which she wrote in collaboration with C. Gogolak and R.
480 Meck. V. Lloyd also wrote an additional paragraph to insert in Section 3.2. S. Hay will
481 incorporate both these additional paragraphs in the next draft of MARSAME.

482 The Workgroup revisited the “Reference Coordinate System” and “Health and Safety” issues
483 highlighted in lines 366-368 of Chapter 3 and decided and that these issues could be deleted and
484 would not be addressed in MARSAME.

485 There was continued discussion on the table proposed in Section 3.7 and whether the Workgroup
486 envisions a table similar to the one in MARSSIM (i.e., Table 4.1) and whether such a table
487 belongs in Chapter 3 or elsewhere in MARSAME. The Workgroup decided to insert this table in
488 Section 3.7 for now and to revise it to include information from MARSSIM Table 4.1 and Table

489 B-3a/b from NUREG 1761 (a copy of which was provided by G. Powers at this meeting). S. Hay
490 was asked to combine the relevant information from both these tables to create a new table for
491 insertion in Section 3.7.

492 Addressing Homeland Security and Scenario B Issues

493 The Workgroup recognized that homeland security issues are summarily addressed in the section
494 on instrument selection, but should also be addressed under the section on survey techniques.
495 The Workgroup also suggested that homeland security be introduced and discussed in the section
496 on release limits. R. Meck reiterated the importance of clarifying the applicability of homeland
497 security issues in the scanning of equipment and materials, e.g., for illegal trafficking. However,
498 for purposes of MARSAME, the scope of the homeland security discussion should be limited to
499 show applicability in terms of radiation detection instruments.

500 MARSAME will need to emphasize Scenario B, which addresses homeland security issues and is
501 not covered in MARSSIM. However, the Workgroup agreed that the surveying framework is the
502 same for both manuals since the survey process will not be affected by the scenario under
503 investigation. Whether the survey involves a detection point for the release of materials or an
504 accident or terrorist attack that requires cleanup / post-RDD cleanup, the survey method will not
505 be altered. Since MARSAME will contain lengthy discussions on new kinds of monitoring and
506 survey instruments, the Workgroup decided to put all these discussions in the Appendix.

507 Review of Chapter 4 of MARSAME

508 Consistency of Terminology and Definitions

509 Where possible, the Workgroup asked the writers to avoid the use of the word “contamination”
510 and its derivatives by using “radiologically controlled” when referring to areas and “residual
511 radioactivity” when referring to equipment and materials. However, since DOE and NRC
512 interpretations of the term “radiologically controlled” differ, the Workgroup advised writers to
513 use the terms “radiologically controlled” and “impacted areas” to accurately define specific
514 situations. The Workgroup endorsed R. Coleman’s use of the term “difficult-to-access” to
515 substitute for the term “inaccessible” and suggested that all the writers use this revised
516 terminology throughout the document.

517 The Workgroup recommended the use the term “clearance” as opposed to “radiological release.”
518 At this point, R. Meck reiterated the distinction between “release” and “clearance” as these terms
519 are understood by the international scientific and regulatory community, i.e., “clearance” applies
520 only to licensed operators while “release” applies to all other activities. Therefore, in the
521 international scientific and regulatory community, “release” is a broader concept that
522 encompasses “clearance.” However, the Workgroup decided that for purposes of MARSAME,
523 “release” will be used to incorporate “clearance” and this distinction will be made in the glossary.

524 The Workgroup modified existing MARSSIM definitions to reach consensus on the definitions
525 of the following key terms for MARSAME:

- 526 • Impacted area - any area that is not classified as non-impacted. Areas with a reasonable
527 possibility of containing residual radioactivity in excess of natural background or fallout
528 levels.
- 529 • Clearance - release of materials from regulatory control (does not apply to terrorist acts).
- 530 • Release - removal of materials from administrative, institutional, or regulatory
531 radiological control.

532 The Workgroup discussed the interchangeable use of the terms “release criteria” and
533 “radiological criteria” in MARSAME and decided that these terms should be replaced by
534 “release limit,” where appropriate. The Workgroup also agreed that it must be emphasized in
535 both the text and the glossary that “release limit” refers to a *radiological* release limit. After
536 reaching this decision, the Workgroup realized that the term “detection limit” is also used in
537 Scenario B cases. To resolve this discrepancy, the Workgroup decided that for purposes of
538 MARSAME, everything is captured under the term “release limit.” This distinction will be
539 clarified in the glossary.

540 The Workgroup consulted the National Research Council’s definition of “surface contamination”
541 as published in “The Deposition Dilemma,” to derive a corresponding definition in MARSAME.
542 The Workgroup decided that “surface contamination” will be referred to as “surficial
543 radioactivity” in MARSAME. The existing MARSSIM definition for “surface contamination”
544 was then modified to arrive at the following definition for “surficial radioactivity” in
545 MARSAME:

- 546 • Surficial radioactivity - residual radioactivity found on surfaces and expressed in units of
547 activity/surface area (Bq/cm² or dpm/100 cm²).

548 By making this change, the Workgroup recognized that the use and definition of the above term
549 will differ from MARSSIM, giving rise to an internal inconsistency between MARSSIM and
550 MARSAME, which is a supplement to MARSSIM. However, the Workgroup envisions this to
551 be short-term issue since it is expected that MARSSIM will also be revised in the future to reflect
552 the altered definition.

553 The Workgroup also consulted the ANSI/HPS (1999) definition for “volumetric contamination”
554 to derive a corresponding definition in MARSAME. The existing MARSSIM definition for
555 “volumetric contamination” was then modified to arrive at the following definition for
556 “volumetric radioactivity” in MARSAME:

- 557 • Volumetric radioactivity - radioactive material residing in or throughout the volume of an
558 item, may result from neutron activation or from the penetration of radioactivity into
559 cracks or interior surfaces within the matrix of an item or solid state diffusion.

560 After discussing the ANSI/HPS (1999) definition's applicability to both MARSAME and
561 MARSAS, the Workgroup decided to revise the above definition as follows for use in the
562 supplements:

- 563 • Volumetric radioactivity - residual radioactivity residing in or throughout the volume of a
564 solid, liquid, or gas.

565 In response to R. Coleman's request for clarification, the Workgroup discussed the use of the
566 terms "direct measurement," "*in-situ* measurement," and "*in-toto* measurement" to ensure that
567 the context and nature of their use in Chapter 4 were consistent with the MARSSIM definitions.
568 Based on the Workgroup's input, R. Coleman will review the use and interrelatedness of the
569 above terms to ensure that they are used correctly in Chapter 4 and are consistent with the
570 MARSSIM definitions.

571 R. Coleman then defined his use of the terms "investigation level" and "action level" in Chapter
572 4 and asked for clarification from the Workgroup regarding these definitions. R. Coleman
573 defined "investigation level" as an alert mechanism and "action level" as an indicator at which
574 action must be taken to correct an error. C. Petullo will review the use of the term "investigation
575 level" in MARSSIM to see if it is consistent with R. Coleman's definition above.

576 Extended Survey Design Information in Appendices

577 R. Coleman inquired whether MARSAME should present rigorous survey design and structure
578 information in Chapter 4. The Workgroup decided that this kind of information may be
579 misinterpreted as mandatory requirements. To prevent it from being too prescriptive and to
580 encourage its use as a road map for all radiological surveys, the Workgroup decided that
581 MARSAME will present complicated scenarios in the Appendices that can be consulted when
582 designing a survey.

583 Discussion on "Smears" in Section 4.3.1.3

584 The Workgroup suggested that the discussion on "smears" in Section 4.3.1.3 should be expanded
585 to emphasize that smears may be useful as semi-quantitative tools but cannot be used exclusively
586 as release criteria. The Workgroup suggested an explicit statement emphasizing that smears
587 cannot be used as stand-alone release criteria, except in a few cases with special circumstances.

588 Quantitative/Statistical Discussions for Chapter 4

589 R. Meck inquired whether, in the context of background radiation measurement variations,
590 homeland security and interdiction issues will be discussed in Section 4.3.5. He asked if
591 background levels will be determined through human measurements or through algorithms for
592 homeland security scenarios. C. Gogolak will address the above issues and develop
593 quantitative/statistical discussions for incorporation into Chapter 4.

594 Future Direction of Chapter 4

595 For the next draft, Chapter 4 will be revised to include more information on conducting specific
596 surveys. S. Hay suggested the inclusion of more prescriptive information to guide users in
597 designing and developing surveys. To accomplish this, he proposed expanding the discussions
598 on the flow charts that were created at the August 2003 MARSSIM Workgroup meeting. The
599 Workgroup will review the flow charts and be prepared to discuss them at the next day's
600 (Thursday, September 25) meeting.

601 S. Hay and R. Coleman noted that certain sections of Chapters 3 and 4 overlapped. Therefore,
602 they will collaborate to consolidate these chapters, where appropriate. The Workgroup members
603 will identify possible areas of overlap and formulate a method for combining these chapters for
604 the next day's (Thursday, September 25) meeting. S. Hay and R. Coleman also volunteered to
605 separately discuss the reorganization of Chapters 3 and 4 after the meeting had adjourned for the
606 day.

607 THURSDAY, SEPTEMBER 25, 2003

608 ATTENDEES:

609	U.S. Environmental Protection Agency, OERR/ERT	C. Petullo
610	U.S. Environmental Protection Agency, ORIA/HQ	K. Klawiter
611	U.S. Environmental Protection Agency, ORIA/HQ	L. Bender
612	U.S. Environmental Protection Agency, ORIA/NAREL	V. Lloyd
613	U.S. Environmental Protection Agency, Region 2	N. Azzam
614	U.S. Nuclear Regulatory Commission, NMSS	J. DeCicco
615	U.S. Nuclear Regulatory Commission, RES	R. Meck
616	U.S. Department of Energy (EM-33)	A. Williams
617	U.S. Department of Energy (EH-41)	E. Boulos
618	U.S. Department of Homeland Security (formerly DOE/EML)	C. Gogolak
619	U.S. Air Force: R. Bhat	

620 MEMBERS OF THE PUBLIC:

621	Oak Ridge National Laboratory	R. Coleman (DOE Contractor)
622		
623	Cabrera Services, Inc.	S. Hay (U.S. Air Force Contractor)
624		
625	SC&A, Inc.	R. Abedin (NRC Contractor)

626 DISCUSSION:

627 Revisions to Flow Charts

628 The Workgroup resumed its discussion on the flow charts that resulted from the August 2003
629 MARSSIM Workgroup meeting. Revisions to these flow charts were intended to provide more
630 detailed discussions in Chapter 4 of the different survey unit scenarios. The Workgroup
631 discussion began with a review of the Class 1 scenario.

632 At this point, C. Petullo directed the Workgroup to focus on other agenda items while K.
633 Klawiter and S. Hay met separately to create detailed flow charts based on the Workgroup's
634 input on the Class 1 survey unit scenario. N. Azzam was tasked with reproducing K. Klawiter's
635 handwritten flow charts in PowerPoint so that these revised flow charts could be presented to the
636 Workgroup for further discussion at the next day's (Friday, September 26) meeting.

637 Reorganization of MARSAME Chapters

638 The Workgroup addressed the reorganization of MARSAME that would result from combining
639 Chapters 3 and 4. E. Boulos distributed a revised outline that he had compiled to address the

640 reorganization of chapters and sections in MARSAME. The Workgroup then combined the
 641 information in Chapters 3 and 4 to arrive at the following strawman outline:

- 642 • Chapter 1 (introductory chapter that has not been written yet)
- 643 • Chapter 2 (“Historical Assessment”) - at the end of this chapter, reader must have
 644 adequate information to determine whether his/her survey unit is impacted or non-
 645 impacted.
- 646 • Chapter 3 - this new chapter will likely be titled “Final Status Survey Considerations,
 647 Planning, and Design” and will be a combination of the current Chapter 3 (“Preliminary
 648 Survey Considerations”) + Chapter 4 (“Survey Planning and Design”).
- 649 • Chapter 4 - this new chapter will discuss survey design, providing descriptions on what
 650 will be measured and how these measurements should be recorded based on the survey
 651 unit determination.
- 652 • Chapter 5 - this chapter will address data quality issues; discuss volumetric vs. surface
 653 issues; evaluate the survey design; identify data quality objectives (DQOs); verify the
 654 data used; and teach the reader how to determine whether the survey followed a
 655 technically defensible approach.

656 In reviewing the strawman outline above, the Workgroup decided that there was a need for
 657 supplemental information in the form of an “implementation” chapter that needs to be inserted
 658 between Chapter 4 (the planning phase) and Chapter 5 (the assessment phase). Further scrutiny
 659 of the strawman outline brought about the question of whether the Workgroup should address
 660 survey design from a “type of survey, i.e., scan only, survey only, or scan + survey” approach
 661 rather than a “type of radioactivity, i.e., surface vs. volumetric (bulk)” approach. Currently,
 662 MARSAME discusses survey design using the following two-pronged approach:

<u>Surface</u>	<u>Volumetric (bulk)</u>
Scan only	Scan only
Scan + Static	Scan + Static
<i>In-toto</i>	<i>In-toto</i>

668 R. Coleman and S. Hay proposed a variation to the above approach and asked the Workgroup to
 669 consider using the following three-pronged approach:

<u>Scan only</u>	<u>Scan + Static</u>	<u><i>In-toto</i></u>
Surface	Surface	Surface

672	Volumetric (bulk)	Volumetric (bulk)	Volumetric (bulk)
673 674 675 676 677 678	<p>The Workgroup discussed whether MARSAME’s revised Chapter 3 should replicate some of the issues covered under MARSSIM’s Chapter 4, e.g., radionuclides of concern, measurement techniques, survey instruments, reference areas, survey units, and classification. The Workgroup decided that these issues should be addressed in the revised Chapter 3, along with conveyerized survey measurements and other innovative measurement techniques that had not been previously discussed in detail in MARSSIM.</p>		
679 680 681 682 683	<p>In reviewing the various issues that would be covered under the revised Chapter 3, the Workgroup realized that it will be a long chapter and a seamless transition between Chapters 3 and 4 will be needed to avoid possible overlap. C. Gogolak suggested using the seven-step DQO process to revise the strawman outline and provide a familiar framework for the users. Using the DQO process, the Workgroup developed the following content for the MARSAME chapters:</p>		
684 685 686 687 688 689 690 691 692	<ul style="list-style-type: none"> • Chapter 1 (introductory chapter that has not been written yet) • Chapter 2 - <i>Define the Problem + Identify the Decision (DQO step #s 1, 2)</i> • Chapter 3 - <i>Inputs to the Decision Process (DQO step # 3)</i> - DCGL, release limit, radionuclides of concern, survey instruments, reference areas • Chapter 4 - <i>Defining the Boundaries (DQO step # 4)</i> - survey units, classification, accessibility - site preparation • Chapter 5 - <i>Decision Rules and Errors (DQO step #s 5, 6, 7)</i> - survey design • Chapter 6 - implementation, quality control, health and safety issues • Chapter 7 - data quality assessment, interpretation of survey results 		
693	<p>Based on this revised strawman outline, the following writing assignments were made:</p>		
694 695 696 697	<ul style="list-style-type: none"> • Chapters 3 and 4 - R. Coleman • Chapters 2 and 5 - S. Hay • (Chapter 6 - on hold for now) • Chapter 7 - C. Gogolak 		
698 699	<p>C. Petullo requested that the writers of Chapters 2, 3, 4, and 5 have their drafts ready for review at the next MARSSIM Workgroup meeting during December 8-12, 2003.</p>		

700 Survey Design Scenarios or “School Problems”

701 In addition to reviewing the above chapters at the upcoming December 2003 meeting, the
702 Workgroup will also review examples of hypothetical survey design scenarios that will be
703 included in the MARSAME Appendices. C. Petullo envisioned that a whole day of the meeting
704 agenda could be committed to discussing these survey design scenarios. A. Williams was tasked
705 with formulating different survey design scenarios or “school problems” involving different
706 facilities and radionuclides.

707 Review of Chapter 7 (formerly Chapter 5/6) of MARSAME

708 C. Gogolak introduced the review of this chapter by providing some historical context on
709 MARSAME’s development as a multi-agency effort. He explained that NUREG 1761, a survey
710 design document, was developed by NRC for its licensees to release materials. To address the
711 release of materials and equipment under the jurisdiction of other Federal agencies, the
712 MARSSIM Workgroup began developing MARSAME. Therefore, Chapter 7, as it currently
713 stands, is mainly a compilation of NUREG 1761 and relevant sections of MARSSIM.

714 R. Meck suggested that homeland security and Scenario B issues, which C. Gogolak had
715 discussed in his presentation at the Health Physics Society (HPS) meeting, be incorporated into
716 the expanded scope of Chapter 7. C. Gogolak agreed to do so but reiterated that these issues
717 need to be first introduced in the discussion on survey design in Chapter 5 and then later re-
718 addressed in Chapter 7 as a check-point to see if the survey process was accurate in its derivation
719 of the DCGL_c.

720 R. Meck inquired whether there was a need for displaying data, e.g., posting plot of results, in
721 cases of release or interdiction. C. Gogolak responded that real-time display of the results and
722 pictures of the vehicles would be helpful, however, unlike NUREG 1761, MARSAME does not
723 require that data be logged at all times. Therefore, without the corresponding data, posting plots
724 would not be possible. C. Gogolak offered to discuss the case of no logged data in Chapter 7
725 since this issue is not addressed in MARSSIM.

726 The Workgroup concurred that the ANSI (E4) reference, which is an interagency (EPA, DOE,
727 and DOD) quality assessment project plans document, should be cited in Chapter 7. C. Gogolak
728 will incorporate discussions on the ANSI document in Chapter 7. C. Goglak then explained to
729 the Workgroup that unlike in MARSSIM, Chapter 7 of MARSAME will depend heavily on
730 MDCs and Scan MDCs, and not as much on WRS and Sign tests. Therefore, the critical
731 elements that need to be discussed in this chapter are the calculation of MDCs and how
732 uncertainty can affect them. C. Gogolak stated that he will expand on the discussion of MDCs
733 and Scan MDCs and provide examples to clarify these issues. Additionally, C. Gogolak will
734 develop the concept of MQC in this chapter. He will discuss that MDCs are easier to implement
735 because technicians can make the determinations, as opposed to MQCs, which are harder to
736 implement because the data need to be compiled, averaged, and analyzed later.

737 FRIDAY, SEPTEMBER 26, 2003

738 ATTENDEES:

739	U.S. Environmental Protection Agency, OERR/ERT	C. Petullo
740	U.S. Environmental Protection Agency, ORIA/HQ	K. Klawiter
741	U.S. Environmental Protection Agency, ORIA/HQ	L. Bender
742	U.S. Environmental Protection Agency, ORIA/NAREL	V. Lloyd
743	U.S. Environmental Protection Agency, Region 2	N. Azzam
744	U.S. Nuclear Regulatory Commission, RES	G. Powers
745	U.S. Nuclear Regulatory Commission, RES	R. Meck
746	U.S. Department of Energy (EM-33)	A. Williams
747	U.S. Department of Energy (EH-41)	E. Boulos
748	U.S. Department of Homeland Security (formerly DOE/EML)	C. Gogolak

749 MEMBERS OF THE PUBLIC:

750	Cabrera Services, Inc.	S. Hay (U.S. Air Force Contractor)
751		
752	SC&A, Inc.	R. Abedin (NRC Contractor)

753 DISCUSSION:

754 August 2003 MARSSIM Workgroup Meeting - Final Meeting Notes

755 The Workgroup reviewed the August 4-7, 2003 meeting notes and provided comments and
756 corrections. R. Abedin will incorporate these revisions and e-mail the finalized August 2003
757 meeting notes to C. Petullo and R. Meck by Friday, October 3, 2003.

758 Revisions to Flow Charts for the SAB-RAC Presentation

759 The Workgroup resumed its discussion on the flow charts that resulted from the August 2003
760 MARSSIM Workgroup meeting. Prior to the Friday meeting, K. Klawiter had created detailed
761 flow charts based on the Workgroup's input on the Class 1 survey unit scenario. N. Azzam
762 reproduced K. Klawiter's handwritten flow charts in PowerPoint so that the Workgroup could
763 review these revised flow charts.

764 As the Workgroup reviewed the handwritten and PowerPoint flow charts, the members realized
765 the complexity of presenting the Class 1 survey unit scenario. The Workgroup concurred that
766 since some of the Class 1 materials and equipment will be disposed of as radioactive waste, they
767 will not be surveyed for release. Therefore, the majority of release surveys will be conducted for
768 Class 3 materials and equipment. The Workgroup concluded that the Class 3 scenario should be
769 included in the SAB-RAC presentation since the Class 3 example is expected to account for the

770 majority of release scenarios. N. Azzam was tasked with updating and revising the flow charts
771 for presentation to the SAB-RAC.

772 SAB-RAC Presentation and Follow-up MARSSIM Workgroup Meeting

773 The Workgroup's consultation with the EPA-SAB-RAC on MARSAME will take place on
774 Tuesday, October 21, 2003. The workgroup reviewed the draft presentation that C. Petullo had
775 developed-based on the Workgroup's feedback at the Monday, September 22 meeting. The
776 Workgroup members provided further feedback and comments.

777 The Workgroup scheduled to meet at NRC (Rockville, MD) for a de-briefing on the SAB-RAC
778 presentation and discuss the survey design scenarios or "school problems" on Wednesday,
779 October 22, 2003. Of the members who were still present at the meeting, R. Meck, A. Williams,
780 L. Bender, E. Boulos, and C. Petullo indicated that they would be available to meet, while the
781 rest indicated that they were not available or would need to check their schedules to confirm their
782 availabilities. C. Petullo will e-mail the Workgroup with an agenda for this meeting.

783 Definition of "Acceptable Knowledge"

784 K. Klawiter presented a copy of the Federal Register Notice (67 FR 154) that contained the
785 EPA's definition of "acceptable knowledge" and explained that this term could not be used
786 interchangeably with "process knowledge." According to EPA, "acceptable knowledge" is a
787 more inclusive concept than "process knowledge," including historical information and prior
788 measurements and reports.

789 September 2003 MARSSIM Workgroup Meeting - Draft Meeting Notes

790 D. Schneider and R. Abedin will compile the draft meeting notes for the September 22-26, 2003
791 Workgroup meeting and e-mail them to C. Petullo, R. Meck, and V. Lloyd by October 3, 2003.
792 V. Lloyd will review the draft meeting notes, insert comments in bold into the document, and
793 distribute this edited version to the Workgroup for comments.

794 Next MARSSIM Workgroup Meetings

795 The workgroup will meet on October 22 following the SAB meeting on October 21. The agenda
796 for this meeting includes:

- 797 < SAB-RAC "hotwash"
- 798 < Review and Refine Example Scenarios

799 Potential agenda items for the December 8-12, 2003 meeting included:

- 800 • Review of the revised drafts of Chapters 2, 3, 4, and 5

- 801 • Continued discussion on the survey design scenarios or “school problems”
- 802 • Overview of the SAB-RAC comments
- 803 • Refinement of the overall survey scenario flow charts

804 ATTACHMENT 1

805 SEPTEMBER 2003 MARSSIM WORKGROUP ACTION ITEMS

806 *N. Azzam*

- 807 1. Update the flow charts according to the comments of the Workgroup.
- 808 2. Search within EPA (e.g., Superfund site, through ORIA, or other divisions) to determine
809 if there is an existing definition for subsurface or volumetric radioactivity.

810 *E. Boulos*

- 811 1. In conjunction with A. Williams, determine whether funding is available for R. Coleman
812 to complete his chapters.

813 *R. Coleman*

- 814 1. Review the use and interrelatedness of the terms “direct measurement,” “*in-situ*
815 measurement,” and “*in-toto* measurement” to ensure that they are used correctly in
816 Chapter 4 and are consistent with the MARSSIM definitions.
- 817 2. Collaborate with S. Hay to consolidate Chapters 3 and 4.
- 818 3. Revise Chapters 3 and 4 and have them ready for review at the next MARSSIM
819 Workgroup meeting (December 8-12, 2003).

820 *C. Gogolak*

- 821 1. Provide logo from the Department of Homeland Security to C. Petullo.
- 822 2. Provide R. Meck with input to help him direct Harry Chmelynski to proceed with refining
823 his two papers in September.
- 824 3. Develop draft FAQs on (1) the percentage scan-to-release issue, (2) the relationship of the
825 MDC to the MQC, and (3) when scanning isn’t possible or scanning inefficiencies don’t
826 allow scanning to the DCGL. At the October 21 SAB meeting, establish a date for a
827 conference call with the Workgroup to discuss them.
- 828 4. Provide bullet points on the FAQs for the presentation to the SAB on October 21.
- 829 5. Write the MDC appendix and use the MDC/MQC FAQ as a starting point.

- 830 6. Provide a copy of presentation given at the HPS meeting to R. Meck.
- 831 7. Develop the graded approach to recordkeeping within Chapter 5.
- 832 8. Address homeland security (Scenario B) and interdiction issues in the context of
833 background radiation measurement variations and develop quantitative/statistical
834 discussions for incorporation into Chapters 4 and 7.
- 835 9. Discuss the case of no logged data and incorporate discussions on the ANSI (E4)
836 reference document in Chapter 7.
- 837 10. Expand on the discussion of MDCs, Scan MDCs, and MQCs and provide examples to
838 clarify these issues in Chapter 7.

839 ***S. Hay***

- 840 1. Update Chapter 3 based on Workgroup comments.
- 841 2. Incorporate two additional paragraphs in Sections 3.1 and 3.2, which were written by V.
842 Lloyd, C. Gogolak, and R. Meck.
- 843 3. Transfer the current version of Chapter 3 and other relevant files to R. Coleman so that he
844 can combine Chapters 3 and 4.
- 845 4. Revise Chapters 2 and 5 and have them ready for review at the next MARSSIM
846 Workgroup meeting (December 8-12, 2003).

847 ***V. Lloyd***

- 848 1. Review the September 22-26, 2003 draft meeting notes, insert comments in bold into the
849 document, and distribute this edited version to the Workgroup for comments.

850 ***R. Meck***

- 851 1. Based on input from C. Gogolak, direct Harry Chmelynski to proceed with refining his
852 two papers in September.
- 853 2. Prepare a presentation for the Workgroup on the application of NUREG-1640 for the
854 meeting.
- 855 3. Provide a paragraph summarizing the paper on small activity quantities and radionuclides
856 with short half lives to be included in Section 3.5.4, prior to the December meeting.

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C. Petullo

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1. Email the Daubert factors paper to E. Boulos and L. Bender.

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2. Ask the author of the Daubert factors paper to speak to the Workgroup.

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3. Finalize the SAB presentation.

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4. At the SAB meeting on October 21, set a date for a conference call to discuss C. Gogolak's draft FAQs.

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5. Review the use of the term "investigation level" in MARSSIM to see if it is consistent with R. Coleman's definition and use of this term in Chapter 4 of MARSAME.

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6. E-mail agendas to the Workgroup for the October and December 2003 meetings.

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7. Check with D. Caputo to determine whether funding is available for S. Hay to complete his chapters.

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A. Williams

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1. Identify the nuclides associated with each of the scenarios to be presented to the SAB to facilitate the selection of example scenarios to combine.

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2. Provide R. Meck with information and data on incineration.

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3. Compile different survey design scenarios or "school problems" involving different facilities and radionuclides and e-mail them to C. Petullo for discussion at the October and December 2003 MARSSIM Workgroup meetings.

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4. In conjunction with E. Boulos, determine whether funding is available for R. Coleman to complete his chapters.

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Entire Workgroup

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1. Provide C. Gogolak with input on factors that are important with regard to uncertainty for his discussion of MDC development.